Title: MCNP6.1.1 Beta Test Suite Comparisons to Delayed Neutron Measurements of $^{233}$U, $^{235}$U, and $^{239}$Pu at the Royal Military College of Canada.

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Intended for:

- An update to LA-UR-14-25702, which compared the MCNP6.1 simulations to measurements.
- Inclusion in M.T. Andrew’s RMCC PhD dissertation.

Accompanying Materials:

- 6 updated MCNP input decks simulating delayed neutron emissions from SNM.

Date: August 2014
Overview

This brief report contains updated delayed neutron (DN) test suite comparisons of MCNP6.1.1Beta to measurements from $^{233}$U, $^{235}$U, and $^{239}$Pu. MCNP simulations recreate the irradiation of milligram quantities of special nuclear materials (SNMs) in aqueous solutions for 60 s. DN magnitudes and temporal behaviors are recorded with F1 tallies and compared to measurements performed at the Royal Military College of Canada. DN emissions of $^{233}$U, $^{235}$U (contained in Nat. U) and $^{239}$Pu were examined up to 3 minutes after the elapse of irradiation, using the three DN options available in MCNP6.1.1 Beta (DN=model, DN=library, and DN=both). This report also demonstrates the improved agreement between measurements and DN=model simulations in MCNP6.1.1 Beta when compared to MCNP6.1 outputs [1].

Experimentation

Solutions containing $^{233}$U, $^{239}$Pu and natural uranium were prepared from certified reference material standards and further diluted with nitric acid and distilled water. Samples were placed in polyethylene vials before pneumatic transport to an inner SLOWPOKE-2 research reactor irradiation site where they were exposed to a predominately thermal neutron flux for 60 s. After irradiation the samples were sent to an array of $^3$He detectors which recorded the DN emissions as a function of count time for up to 3 minutes. Further details regarding the delayed neutron counting system and these measurements can be found in reference 2. Experimental data has been corrected for dead time effects and neutron background contributions [3]. Measurements have been normalized by fissile mass [g] and detection efficiency (33 %) to obtain DN emission rate, $Q(t)$ [s$^{-1}$g$^{-1}$]. Each isotope ($^{233}$U, $^{235}$U, and $^{239}$Pu) was irradiated and counted in triplicate; the provided measurements represent their average $Q(t)$. Plots with error bars included represent the 95 % confidence interval on measurements.

MCNP Simulations

Atomic Energy of Canada Limited has provided a MCNP input deck containing LEU SLOWPOKE-2 dimension and material specifications, the contents of which are detailed in reference 4. This input deck was modified to include a polyethylene vial within an inner irradiation site to determine a higher fidelity neutron flux spectrum. This flux was recreated within the vial solution of a second input deck, which includes the irradiation of a fissile solution for 60 s and the recording of subsequent DN emissions from the vial. The DN emission rate, $Q(t)$ [s$^{-1}$g$^{-1}$], for each MCNP6 simulation was compared to the normalized measurements described in the previous section. Further details on MCNP simulations are included in Reference 5.
Comparisons

**MCNP6.1.1 beta DN=model, library, and both comparisons**

Figure 1 compares the measurements of DN emissions for $^{233}$U and MCNP6.1.1 beta simulations using the three DN emission options. The DN=both option is omitted in subsequent comparisons because it is indistinguishable from the DN=library option. Figures 2-4 compare the DN emission rate for $^{233}$U, $^{235}$U (in Nat. U), and $^{239}$Pu, using the DN=model and DN=library options in MCNP6.1.1 beta.

**MCNP6.1.1 beta comparisons to MCNP6.1**

Simulations using MCNP6.1 and the DN=model option resulted in deviations from measurements at approximately 100 s because of the time-bin structure used to sample delayed particle emissions [6]. MCNP6.1.1 beta contains updates to the delayed particle time-bin structure [7], which eliminates the 100 s anomaly, shown in Figures 5-7. Figures 8-10 compare DN emission rates using the DN=library option with MCNP6v1 and MCNP6.1.1 beta, which remained the same between versions.

**Summary**

DN emissions from $^{233}$U, $^{235}$U, and $^{239}$Pu were compared to MCNP61.1 Beta simulations using the DN=model, both, and library options. Significant improvements in the agreement with measurements and the MCNP6.1.1 Beta DN=model option are noted when compared to MCNP6.1 simulations.
Figure 1: Delayed neutron emission rates from $^{233}$U measurements and three DN options in MCNP6.1.1 Beta simulations.
Figure 2: Delayed neutron emission rates from $^{233}$U measurements, model and library DN options in MCNP6.1.1 Beta simulations.
Figure 3: Delayed neutron emission rates from nat. U measurements, model and library DN options in MCNP6.1.1 Beta simulations.
Figure 4: Delayed neutron emission rates from $^{239}$Pu measurements, model and library DN options in MCNP6.1.1 Beta simulations.
Figure 5: Delayed neutron emission rates from $^{233}$U measurements, DN=model simulations with MCNP6.1 and MCNP6.1.1 Beta.
Figure 6: Delayed neutron emission rates from nat. U measurements, DN=model simulations with MCNP6.1 and MCNP6.1.1 Beta.
Figure 7: Delayed neutron emission rates from $^{239}$Pu measurements, DN=model simulations with MCNP6.1 and MCNP6.1.1 Beta.
Figure 8: Delayed neutron emission rates from $^{233}$U measurements, DN=library simulations with MCNP6.1 and MCNP6.1.1 Beta.
Figure 9: Delayed neutron emission rates from nat. U measurements, DN=library simulations with MCNP6.1 and MCNP6.1.1 Beta.
Figure 10: Delayed neutron emission rates from $^{239}$Pu measurements, DN=library simulations with MCNP6.1 and MCNP6.1.1 Beta.
Figure 11: Delayed neutron emission rates from $^{233}\text{U}$ measurements, model and library DN options in MCNP6.1.1 Beta simulations. Error bars represent 95% confidence intervals.
Figure 12: Delayed neutron emission rates from nat. U measurements, model and library DN options in MCNP6.1.1 Beta simulations. Error bars represent 95% confidence intervals.
Figure 13: Delayed neutron emission rates from $^{239}$Pu measurements, model and library DN options in MCNP6.1.1 Beta simulations. Error bars represent 95% confidence intervals.
References


Example of an Input Deck
Modeling Delayed Neutron Emissions in RMCC's DNC System
c
-------------------------GEOMETRY--------------------------------------
c
27 5000 -0.9977 -509  $ top smaller vial solution
imp:n=1
c
30 0 509  $ geometry void
imp:n=0
c
---------------------------SURFACE CARDS----------------------------------
c
509 rcc 0 0 17.3 0 0 1.38 0.4826  $ top small vial solution
c
c-----------------------------MATERIAL AND SOURCE CARDS---------------------------
c
mode n
c
ACT
$ change between DN=model, DN=both and DN=library
DNBlas=10  $ biases the # of DNs produced
c
c-----------------------------MATERIAL DEFINITIONS-------------------------------
c
m5000
94239 -2.14e-3  $ 94239 for Pu239, 92233 for U233, 92235 for U235
1001 -0.10531
8016 -0.87777
7014 -0.01478
mt5000 lwtr.10t
c
c-------------------------SOURCE DEFINITION----------------------------------
c
sdef pos=0 0 18.0 par=n cel=27
$ To be used when reproducing flux magnitude
Rad=D2 Ext=D3 AXS 0 1 0
mcg=d4 tme=d1 wgt=1.92e14  $ accounts for flux and mass norms
c
Irradiation Time (shakes)
si1 H 0 60e8
sp1 D 0 1
c
si2 H 0 0.4826
sp2 -21 1
si3 -0.7 0.7
c
Particle Time, Weight and Energy Cut-Offs
.cutn 243e8 j 0 0
c
F1:n (509.1 509.2 509.3)
T1: 63e8 179i 243e8
c
F44:n 27  $ checking the flux distribution
E44: 0.625e-6 0.5 10
T44: 1e8 243e8
c
c si4 & sp4 reproduce the 69 energy group neutron flux of the SLOWPOKE-2 (omitted from report).